



The Generation IV R&D Roadmap Overview

***RELAP5 Users Meeting: Park City, UT
September 4, 2002***

Objective – Next Generation Nuclear Energy

The Technology Roadmap:

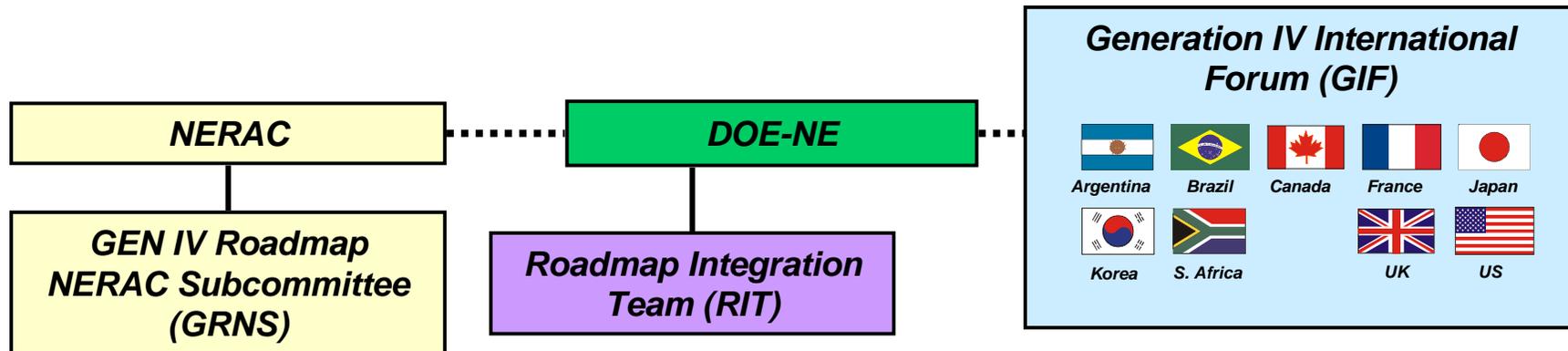
- ***Describes systems deployable by 2030 or earlier***
- ***Determines which systems offer significant advances towards:***
 - ***Sustainability***
 - ***Safety and reliability***
 - ***Economics***
- ***Plans activities for a Generation IV R&D program***

Key Steps to Prepare the Roadmap

- ***Define Technology Goals for Generation IV***
- ***Identify Concepts with Potential***
- ***Evaluate Concepts with a Common Methodology***
- ***Identify R&D Gaps and Needs***
- ***Assemble a Program Plan***

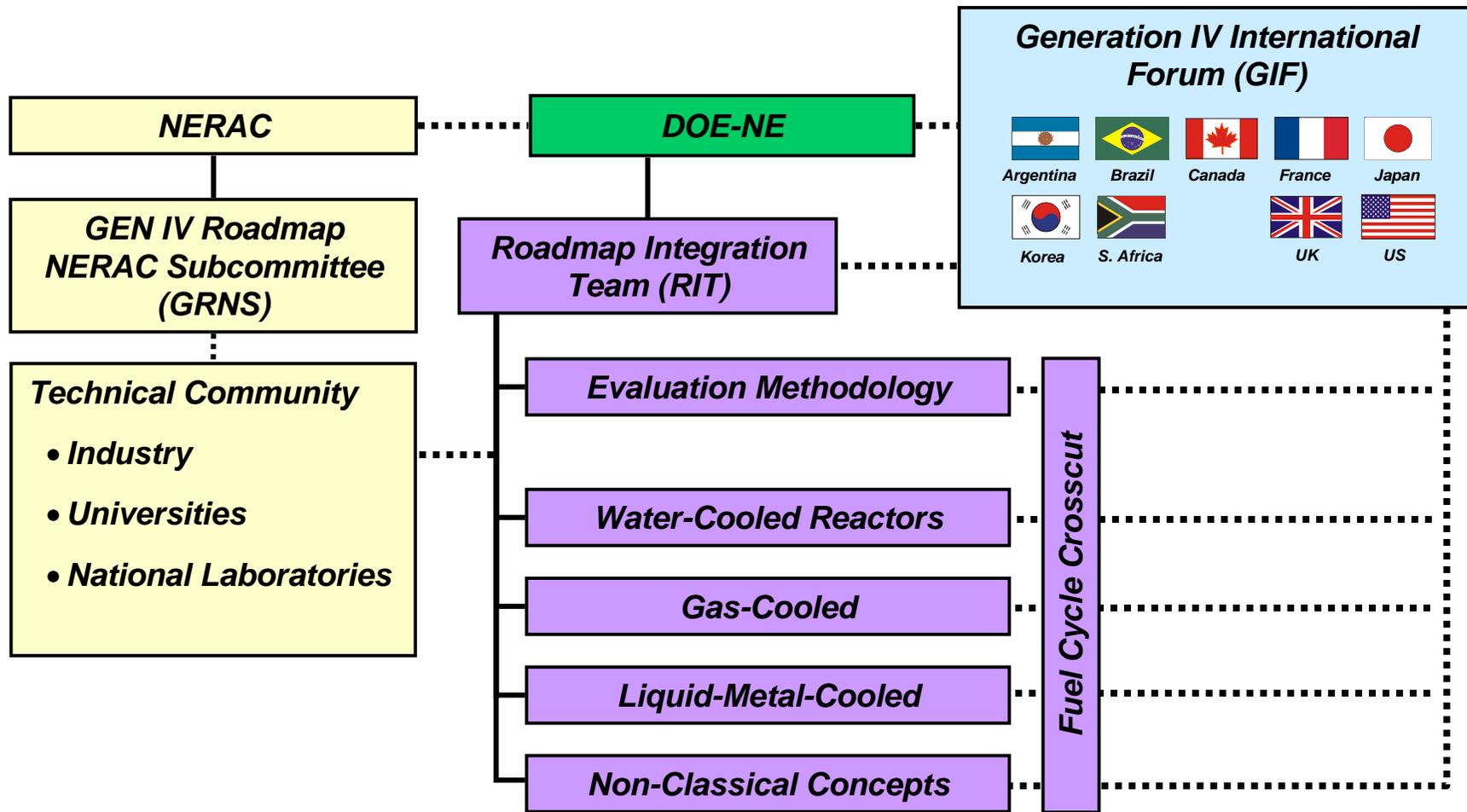
Organizational Evolution

- Jan 2000 First Meeting of 9 Countries on Generation IV
- Sep 2000 Creation of NERAC Subcommittee



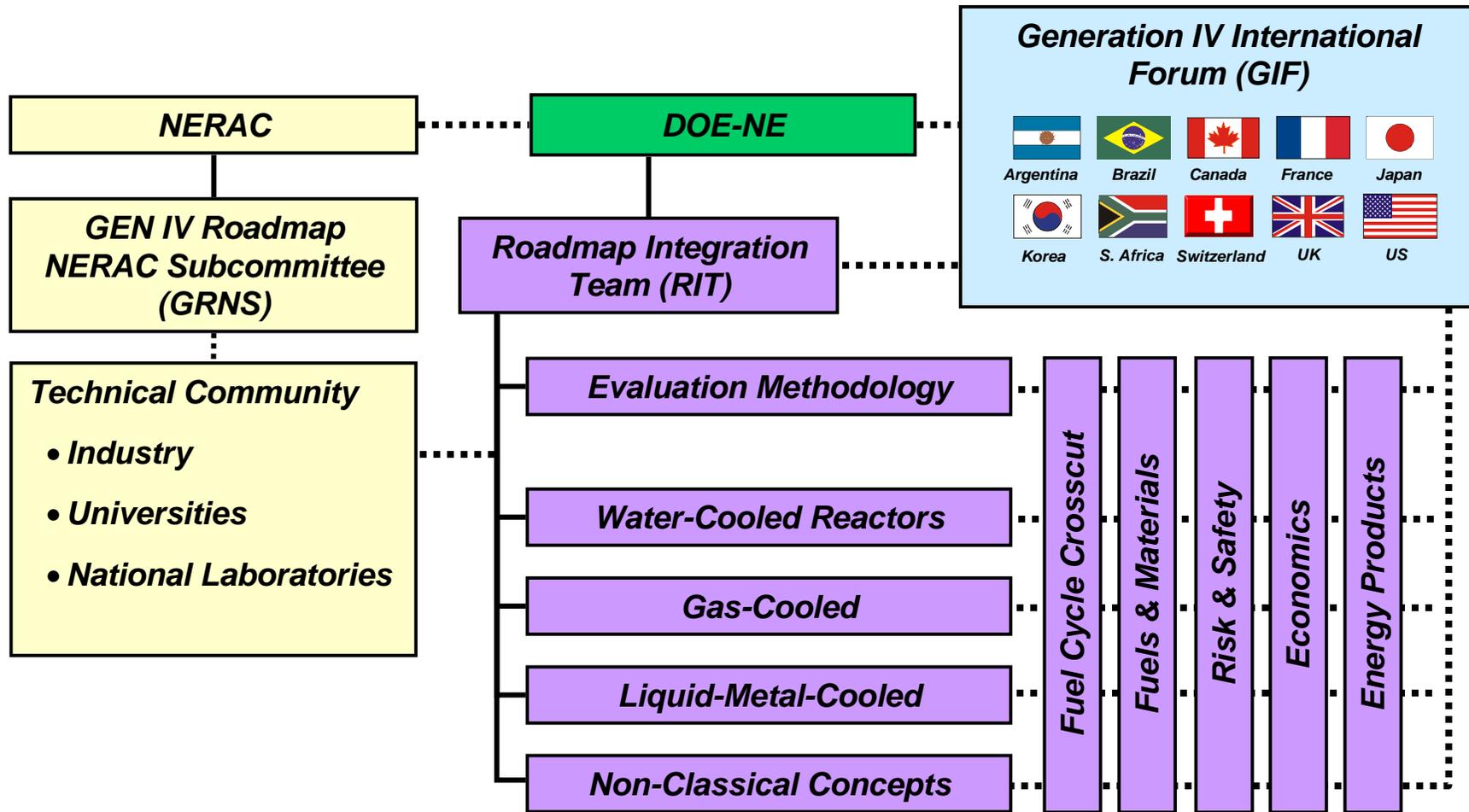
Organizational Evolution

- Dec 2000 Creation of Working Groups and Fuel Cycle Crosscut
- Mar 2001 Incorporation of International Membership



Organizational Evolution

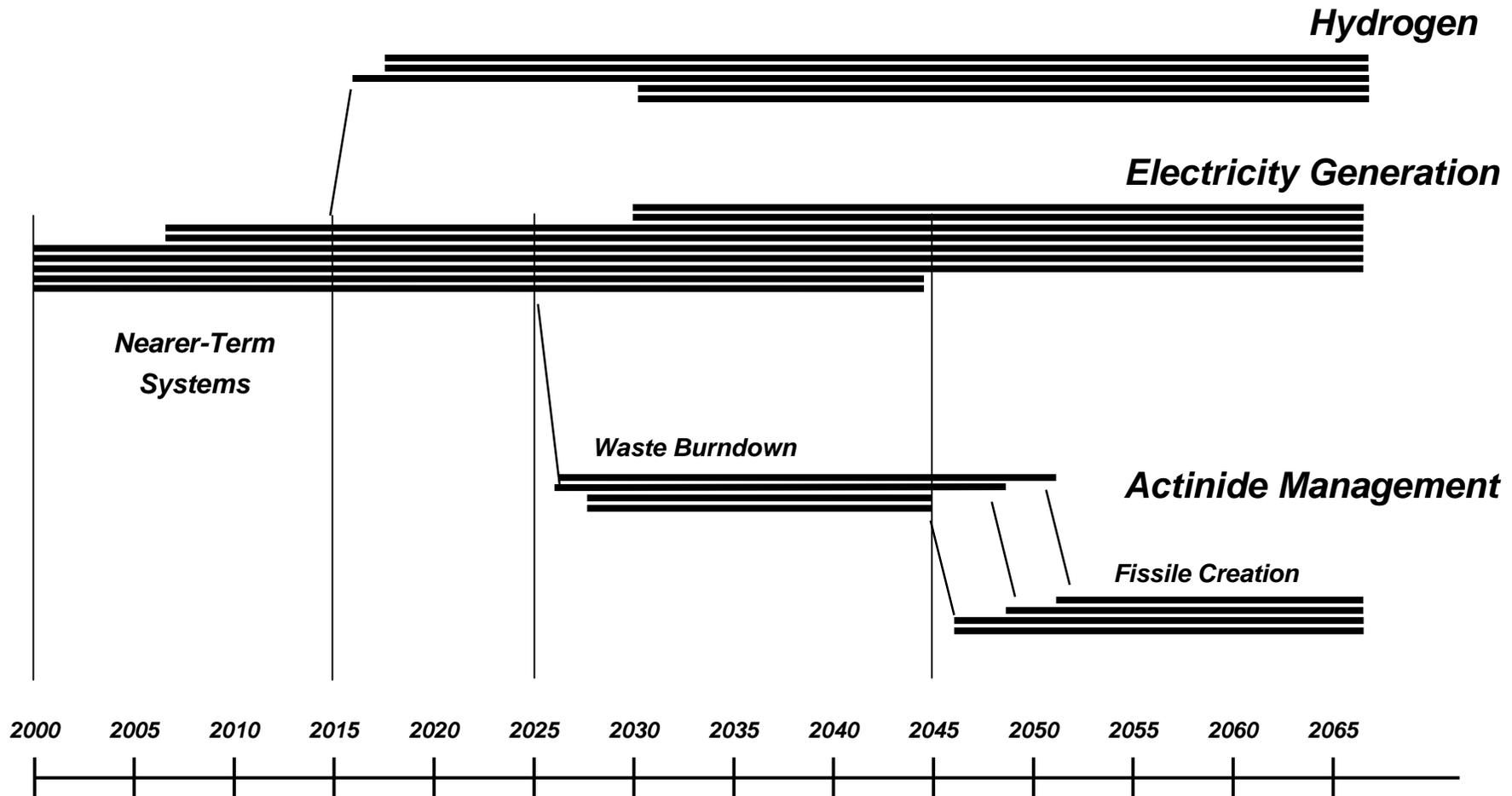
- Sep 2001 Organization of Crosscut Groups
- Feb 2002 Switzerland joins the GIF



GIF Meetings

<i>January 2000</i>	<i>Washington</i>	<i>Countries support Gen IV idea</i>
<i>April 2000</i>	<i>Washington</i>	<i>Experts convened on path forward</i>
<i>August 2000</i>	<i>Seoul</i>	<i>Comment on goals, write charter</i>
<i>March 2001</i>	<i>Paris</i>	<i>Finalize charter, support roadmap</i>
<i>October 2001</i>	<i>Miami</i>	<i>Comment on methodology</i>
<i>February 2002</i>	<i>London</i>	<i>Discuss concepts and selection</i>
<i>April 2002</i>	<i>Washington</i>	<i>Review concept evaluations</i>
<i>May 2002</i>	<i>Paris</i>	<i>Select 6-8 long-term concepts</i>
<i>July 2002</i>	<i>Rio de Janeiro</i>	<i>Review R&D plans</i>
<i>September 2002</i>	<i>Tokyo</i>	<i>Plan R&D collaborations</i>
<i>March 2003</i>	<i>Capetown</i>	<i>Plan R&D collaborations</i>

Important Missions for Generation IV



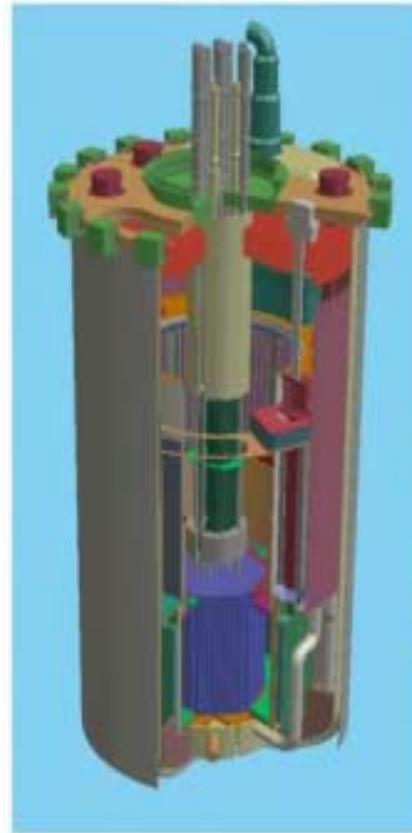
Sodium Liquid Metal-Cooled Reactor (Na LMR)

Characteristics

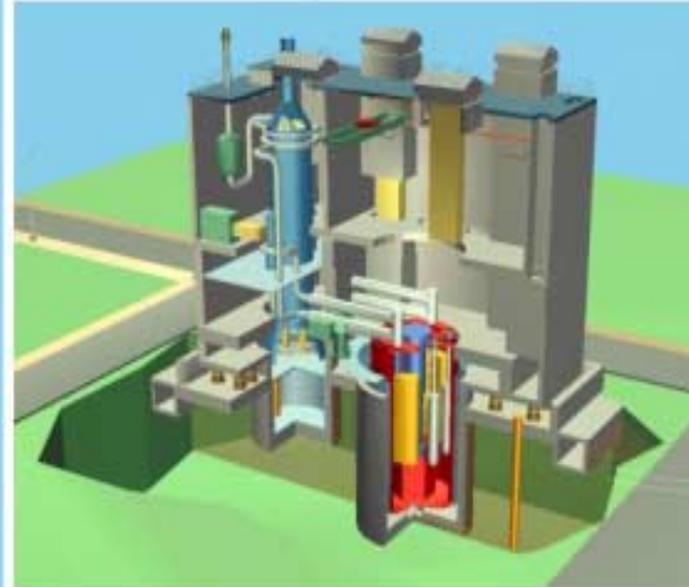
- Sodium coolant, 550°C
- 150 to 500 MWe
- Metal fuel with pyro processing / MOX fuel with advanced aqueous

Benefits

- Consumption of LWR actinides
- Efficient fissile material generation



Liquid Metal Reactor
Actinide Recycle
System



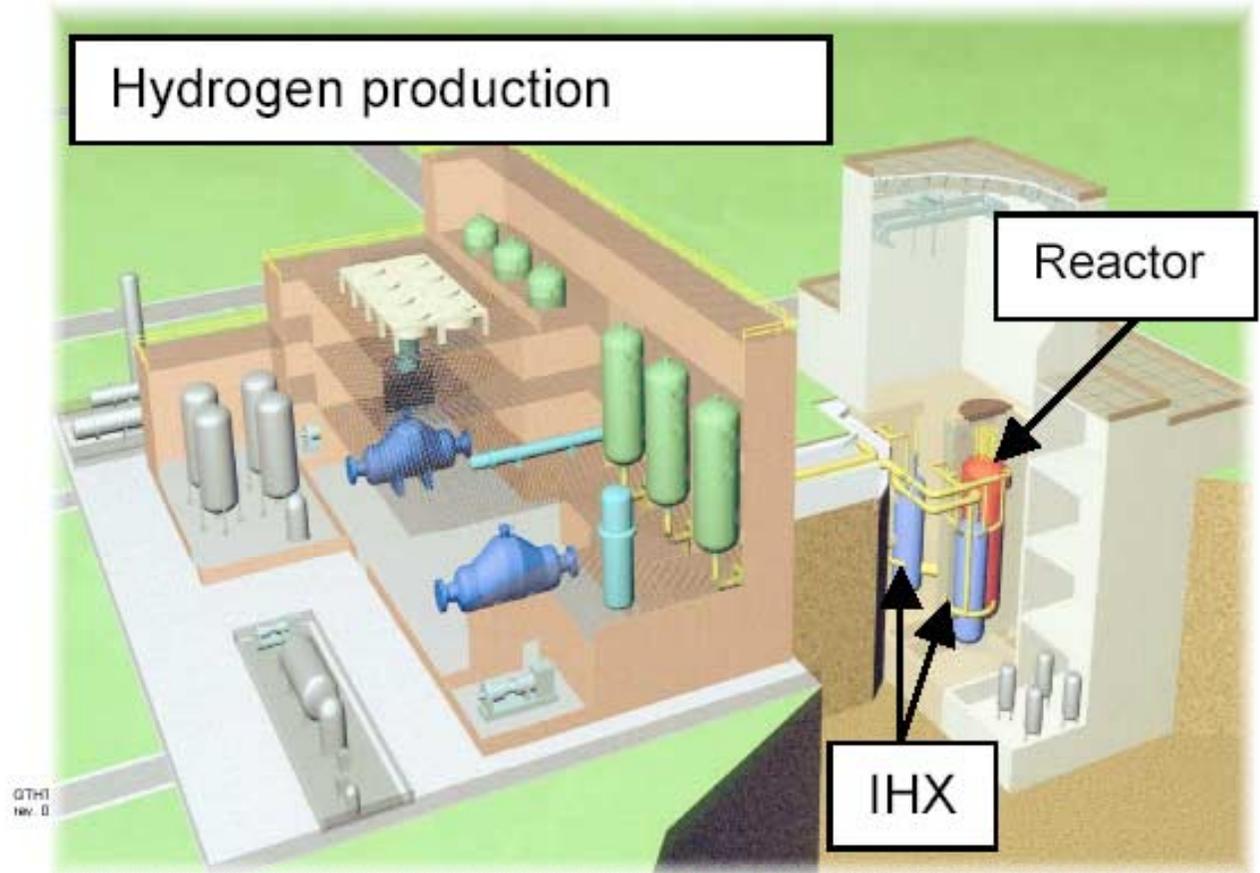
Very High-Temperature Reactor (VHTR)

Characteristics

- He coolant
- 1000°C outlet temperature
- 600 MWe
- Solid graphite block core based on GT-MHR

Benefits

- High thermal efficiency
- Hydrogen production
- Process heat applications
- High degree of passive safety



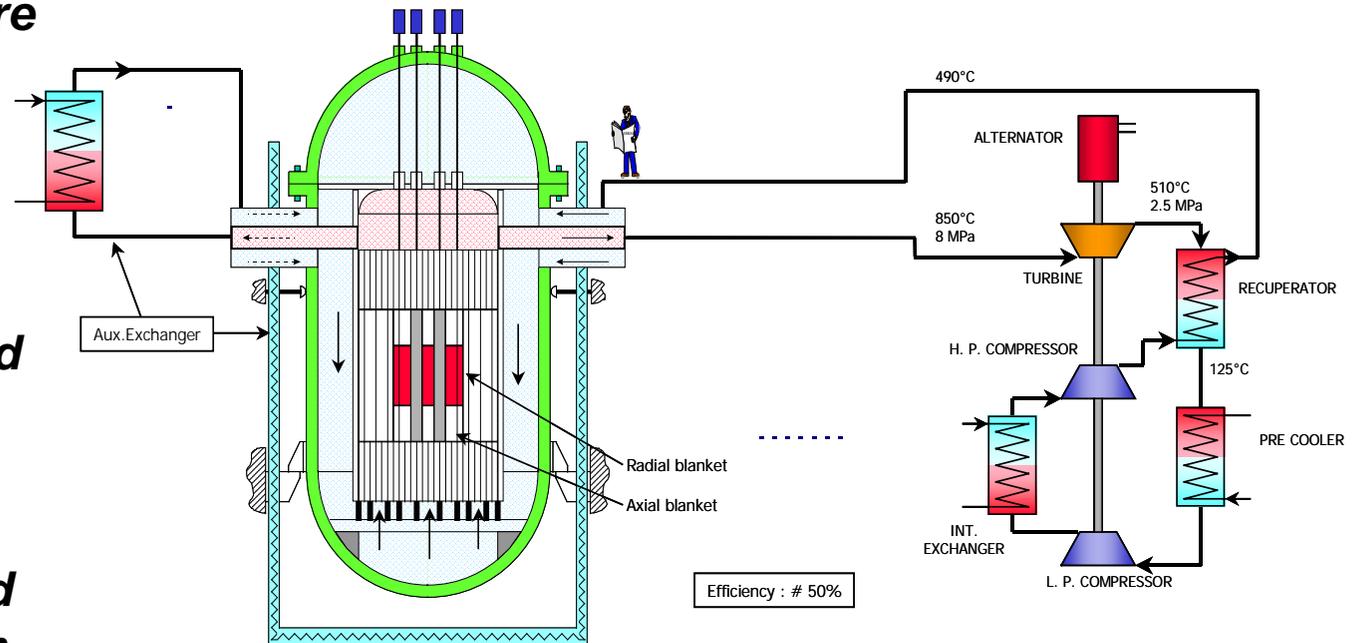
Gas-Cooled Fast Reactor (GFR)

Characteristics

- He coolant
- 850°C outlet temperature
- direct gas-turbine conversion cycle – 48% efficiency
- 600 MW_{th}/288 MW_e
- Several fuel options and core configurations

Benefits

- Waste minimization and efficient use of uranium resources



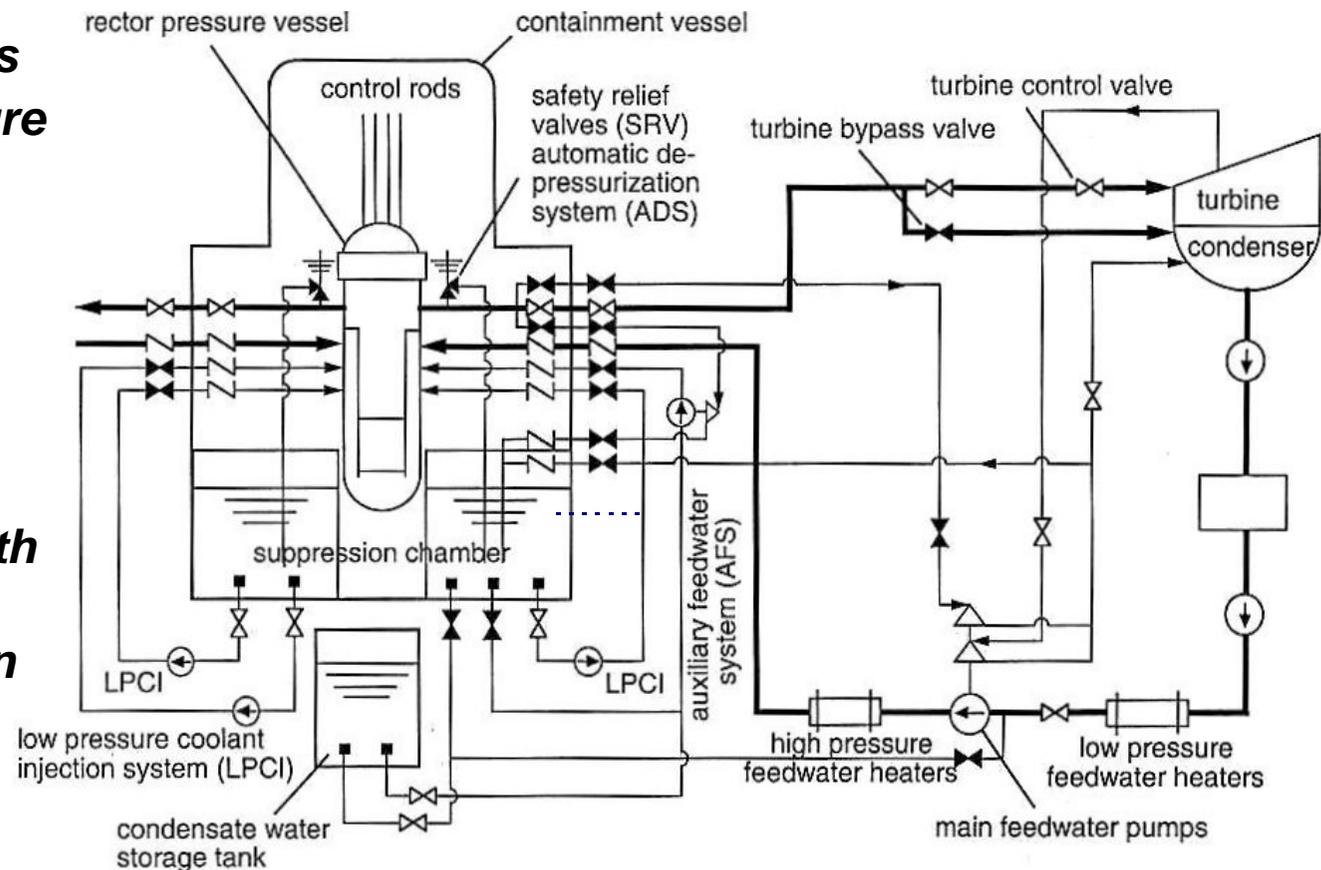
Supercritical Water Reactor (SCWR)

Characteristics

- **Water coolant at supercritical conditions**
- **550°C outlet temperature**
- **1700 MWe**
- **Simplified balance of plant**

Benefits

- **Efficiency near 45% with excellent economics**
- **Thermal or fast neutron spectrum**



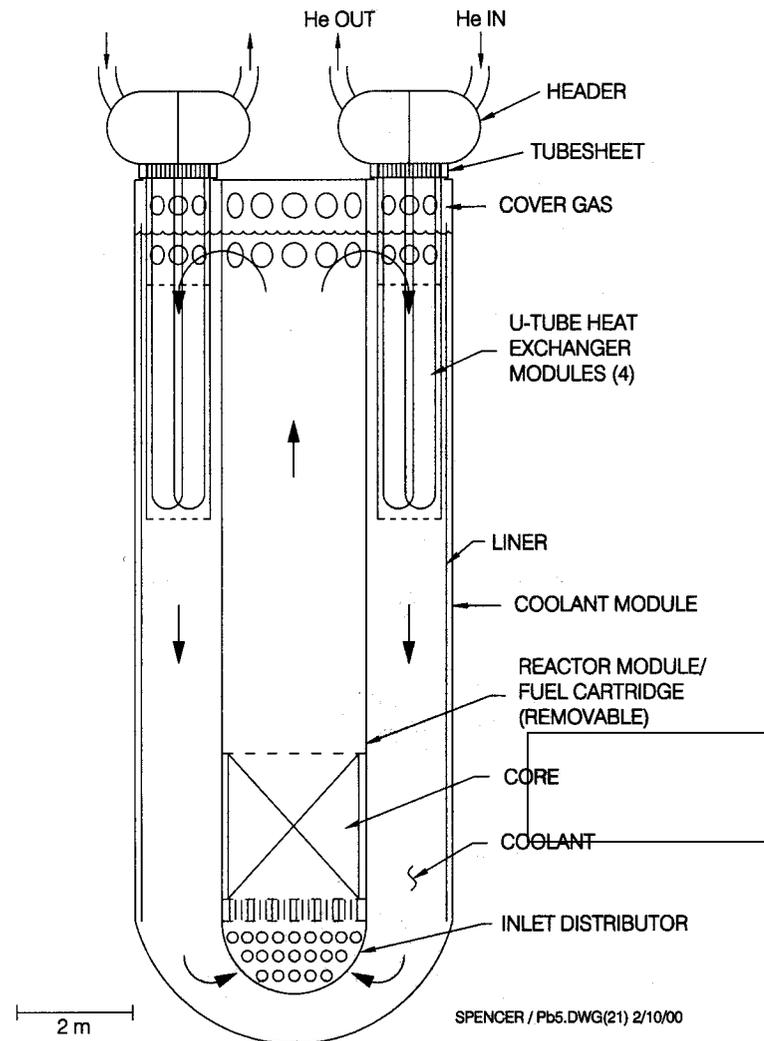
Pb/Bi Reactor – Cartridge Core (Pb/Bi Battery)

Characteristics

- **Pb/Bi or Pb coolant**
- **550°C to 800°C outlet temperature**
- **120-400 MWe**
- **15-30 year core life**

Benefits

- **Distributed electricity generation**
- **Hydrogen and potable water**
- **Cartridge core for regional fuel processing**
- **High degree of passive safety**
- **Proliferation resistance through long-life cartridge core**



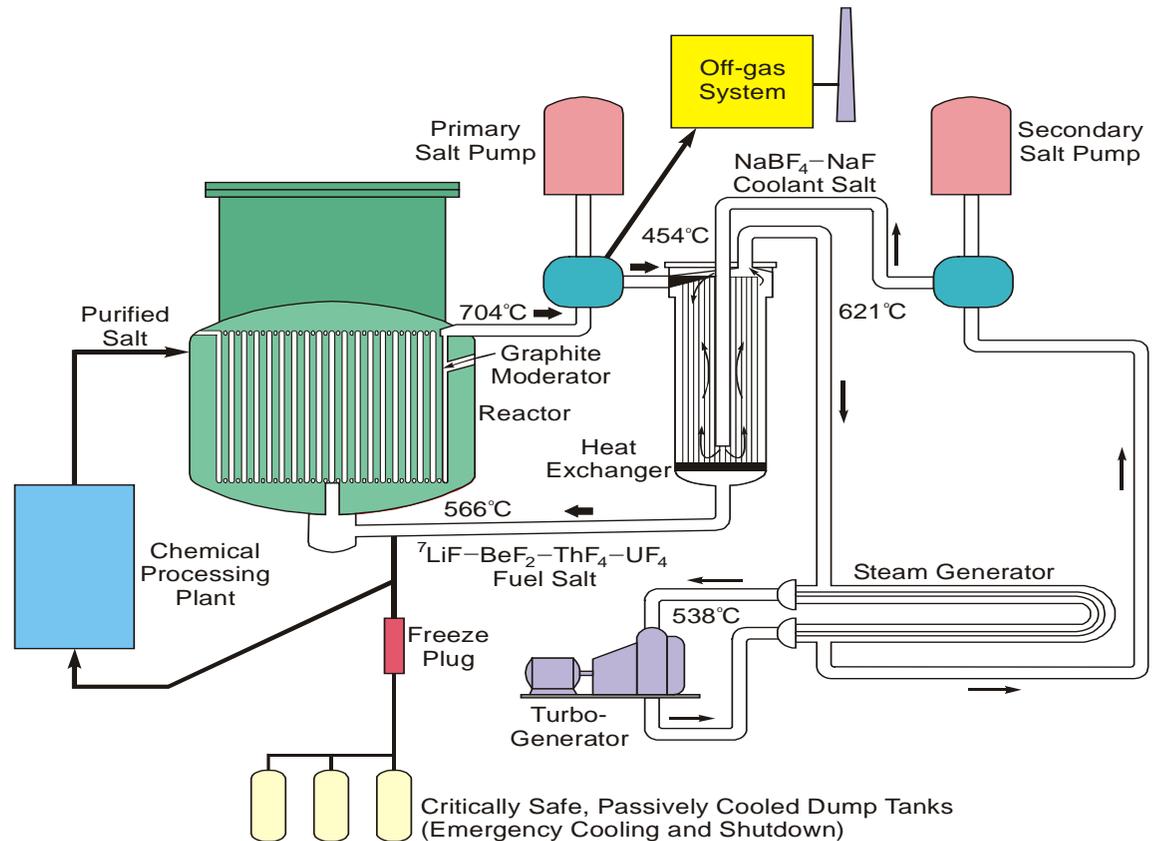
Molten Salt Reactor (MSR)

Characteristics

- **Fuel: liquid U, Pu fluorides, NaF/Zr F₄**
- **700 – 800 °C outlet temperature**
- **1000 MWe**
- **Low pressure (<0.5 MPa)**

Benefits

- **Low source term due to online processing**
- **Waste minimization and efficient use of uranium resources**
- **Proliferation resistance through low fissile material inventory**



Highlights of System Concept Strengths

Sustainability

- ***Closed cycle fast-spectrum systems***
 - ***Reduced waste heat and radiotoxicity***
 - ***Optimal use of repository capacity***
 - ***Resource extension via regeneration of fissile material***

Safety and Reliability

- ***Many concepts make good advances***

Economics

- ***Water- and gas-cooled concepts***
 - ***High thermal efficiency***
 - ***Simplified balance of plant***
 - ***Large and small plant size***

Hydrogen production and high-temperature applications

- ***Temperatures above 700-800°C***

Selected Highlights of the R&D Challenges

- ***Fuels and materials for higher temperatures***
- ***Increased corrosion/erosion in alternative coolants***
- ***Design for inherent safety***
- ***Recycling technology and methods***
- ***Fuel fabrication methods***
- ***Hydrogen thermochemical processes***
- ***Manufacturing and constructability***
- ***Component technologies***